

CTH- Status, plans and opportunities

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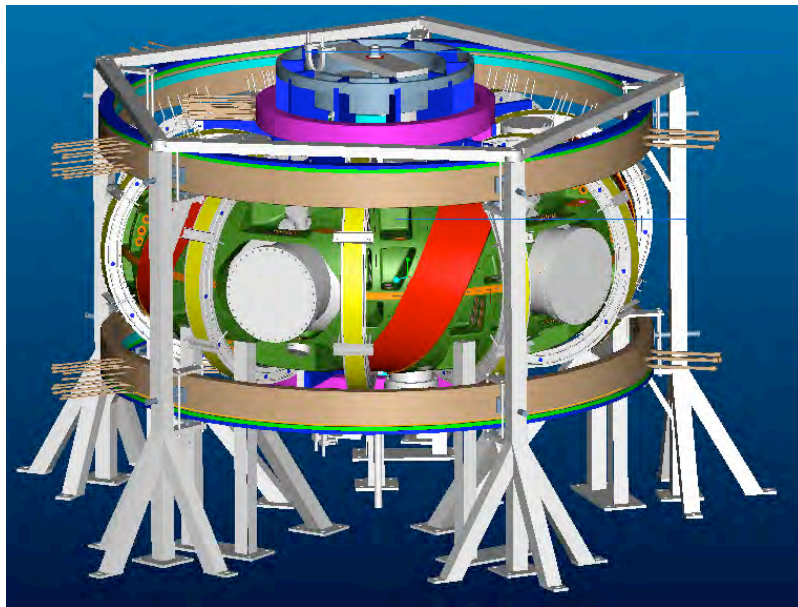
J. Hanson & J. Munoz

May 11, 2006

Advertisement for next US/Japan Stellarator workshop

- Last one was Feb. 2004 in Kyoto
- 2-1/2 day workshop
- Current plan: to be held in Auburn right before or right after DPP meeting in Philadelphia
 - Oct. 25-27 or Nov. 6-8
- Expect about 5 Japanese visitors
- Looking for representation from US stellarator labs & theory groups
 - ~ 25 attendees

Overview of Compact Toroidal Hybrid (CTH)



Areas of investigation

- Effect of controlled static islands
- 3-D equilibrium reconstruction
- disruption avoidance in current-driven stellarators

Torsatron configuration with additional:

- Ohmic plasma current for stability studies
- TF coils for vacuum iota control
- Shaping vertical field (SVF) coils for variation of vacuum shear and elongation
- Set of error correction coils

5 field periods

$$R_o = 0.75 \text{ m}, R/\langle a \rangle \geq 4$$

$$B_o \leq 0.6 \text{ T}$$

$$I_p \leq 50 \text{ kA}; \Delta i \leq 0.5$$

$$P_{in} = 20 \text{ kW ECRH @ 18GHz}$$

$$100 \text{ kW OH}$$

$$\text{Vacuum } i(a): 0.2 - 0.6$$

$$\text{Discharge duration: } 0.5 \text{ s}$$

$$\text{w/ OH: } 0.1 \text{ s}$$

Compact Toroidal Hybrid to contribute to NCSX research

- Similarities to NCSX

- Operation with plasma current; current-driven instabilities, potential for disruptions.
- Strong modification of vacuum equilibrium; need of new reconstruction methods.
- Controllable rotational transform spanning major rational surfaces
- Low aspect ratio

- Key differences with NCSX

- CTH non-symmetric; no optimized confinement or viscosity
- Low- β , low temperature; resistive time scales shorter
- focus on equilibrium & stability
- Scale of effort

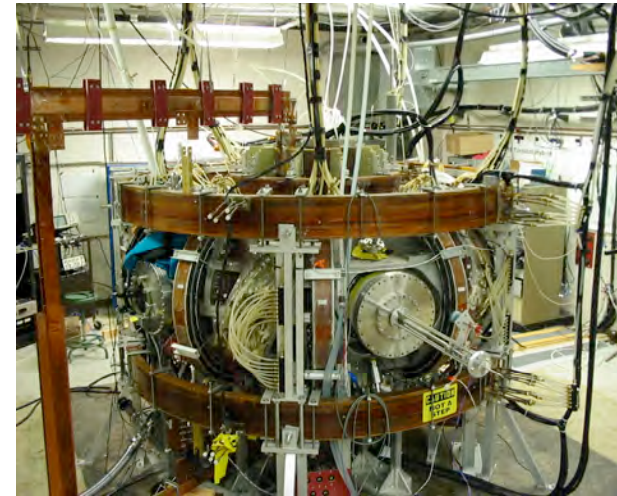
Progress and status

Basic assembly completed Winter 2005

- First plasma
- Initial alignment & field mapping
 - Precision $\Delta x/R \sim 1/500$ sought
 - Measurement of flux surfaces, magnetic islands
 - Initial tests of island correction

Recent installation, alignment, and final mounting of all poloidal coils, including OH stack

- Improved coil positions; repaired shorted coil
- Now field mapping with all equilibrium coils
- Magnetic island control studies continuing
- ECRH ready for 2nd harmonic X-mode heating at 0.33 T, fundamental O-mode at 0.66 T.



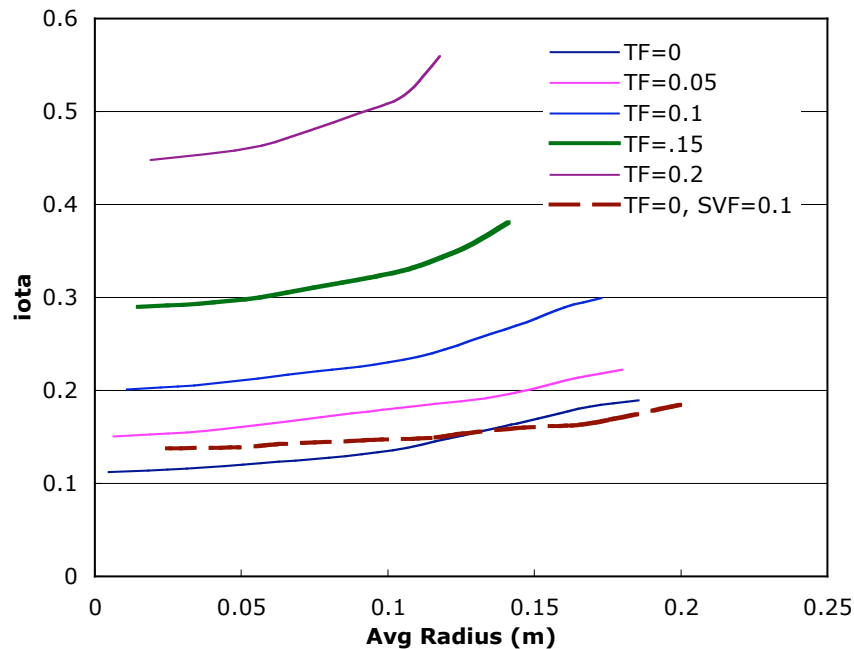
Equilibrium diagnostics

- Ex-vessel magnetics
 - Flux loops, Rogowski coils
- In-vessel magnetics
 - segmented rogowski coils, (B-dot probes)
- Langmuir probes
- SXR arrays (to be installed later)
- mm-wave interferometry/polarimetry (internal B-field diagnostic under scoping devel.)

Stability studies facilitated by flexibility of CTH configuration

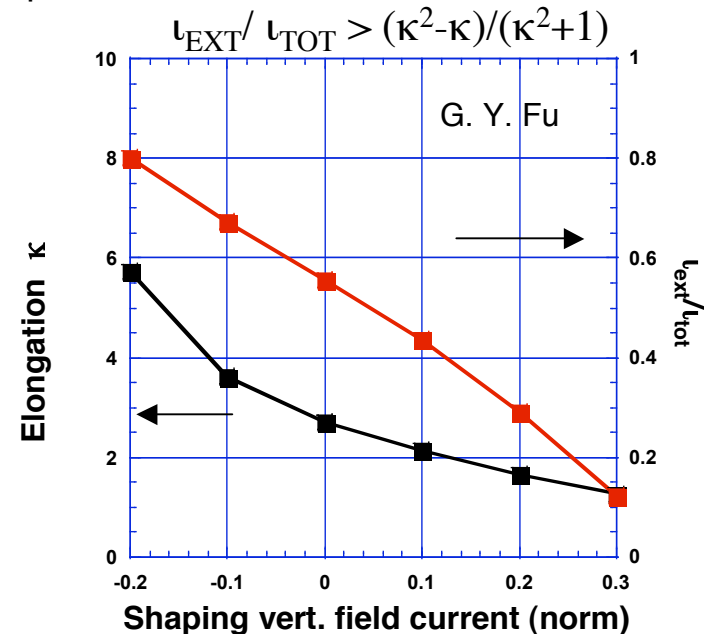
Vacuum rotational transform highly variable with auxiliary toroidal field (TF); can avoid major resonances at edge for range of internal plasma currents

Can independently vary shear in both positive & negative direction



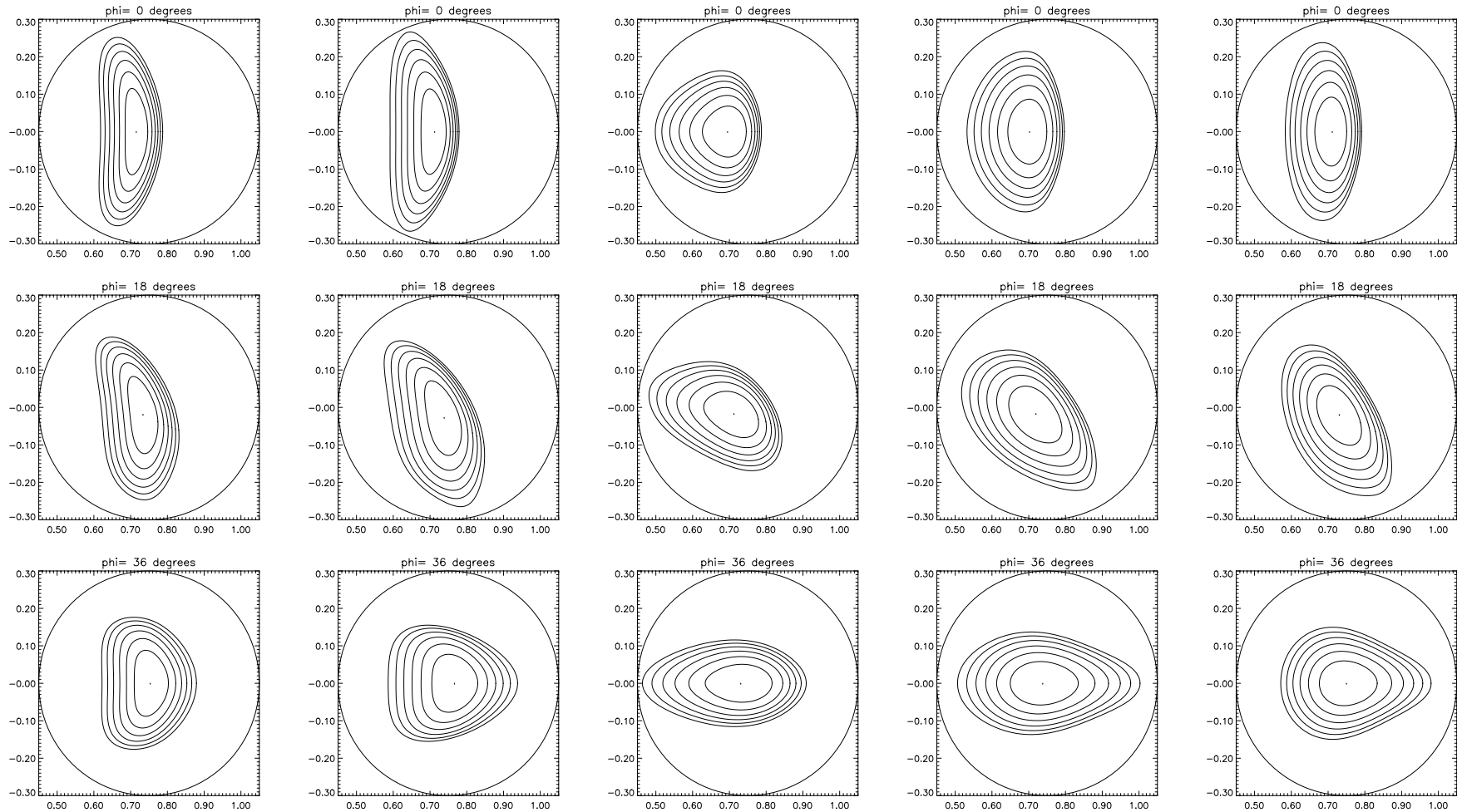
Ideal vertical stability predicted to improve with increasing $\iota_{\text{ext}}/\iota_{\text{tot}}$

Red trace is minimum value of fractional external transform required to stabilize vertical displacement



- Analytic modeling consistent by spot checks with TERPSICHORE ideal MHD stability code
- Ideal kink stability also generally predicted (in 1-D) to improve with increasing $\iota_{\text{ext}}/\iota_{\text{tot}}$

Flexibility of CTH Equilibrium



$I_{TF} = 0.0$
 $I_{SVF} = 0.00$
 $I_p = 0kA$

$I_{TF} = 0.26$
 $I_{SVF} = 0.00$
 $I_p = 0kA$

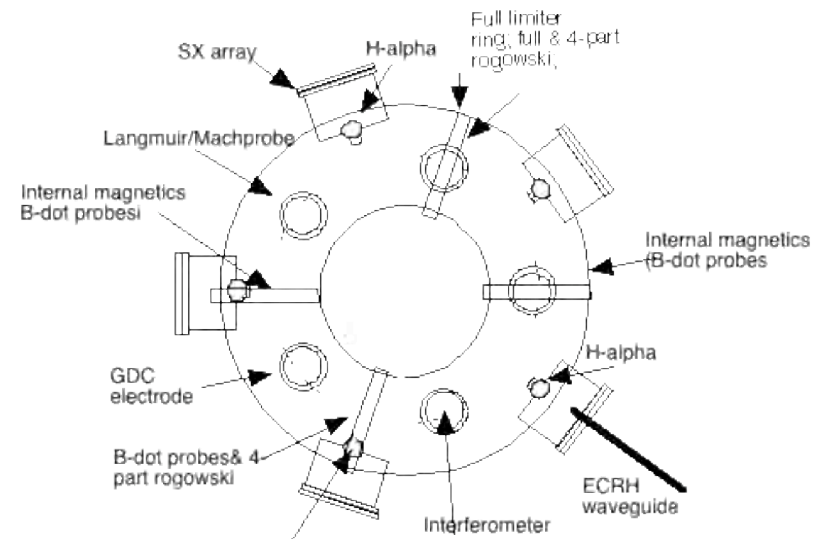
$I_{TF} = 0.13$
 $I_{SVF} = 0.48$
 $I_p = 0kA$

$I_{TF} = 0.13$
 $I_{SVF} = 0.0$
 $I_p = 60kA$

$I_{TF} = 0.13$
 $I_{SVF} = 0.48$
 $I_p = 60kA$

Shaping flexibility => magnetic diagnostics far from plasma

- difficult to infer characteristics of current profile
- Initial in-vessel magnetic diagnostic consists of 2 segmented (4-part) rogowski coils at symmetry planes ($\phi = 0$ and π/N) to measure current moments, as in W7-AS
- Plans for internal B measurement with Faraday rotation; should try shielded probes inserted into low density discharge in near future.



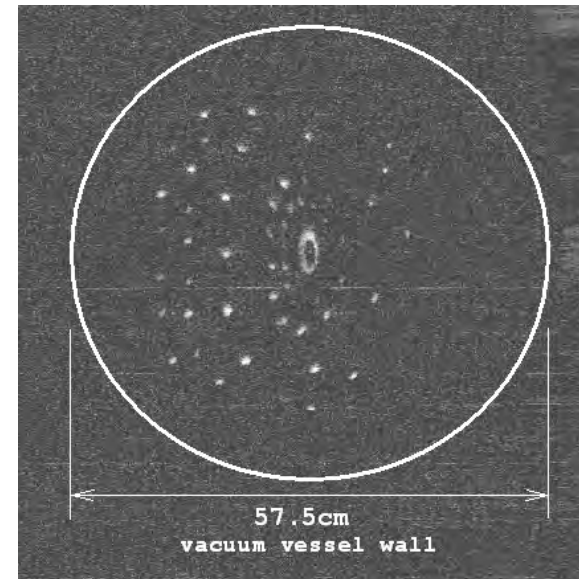
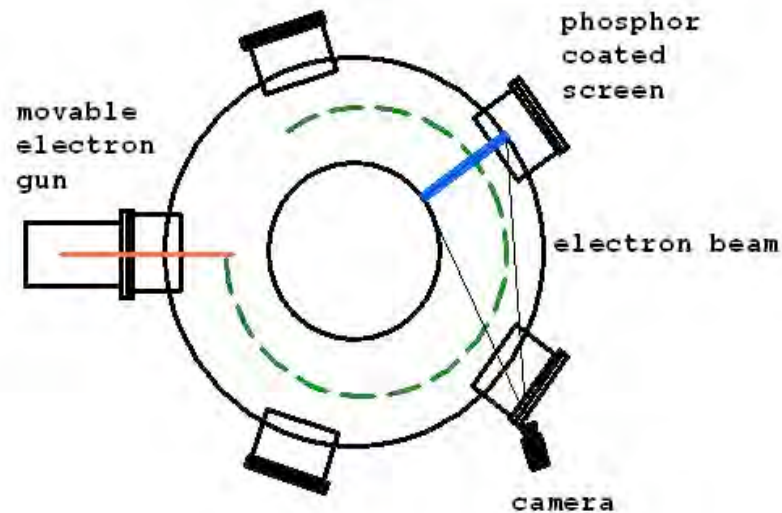
Measurement of vacuum flux surfaces

Objectives

- Identify bad or misplaced coils
⇒ hopefully doesn't **require** field mapping...
- Comparison with design flux surfaces to update model of vacuum field for equilibrium reconstruction
⇒ Improved coil model; in situ calibration of magnetic diagnostics
- Provide basis for implementing island correction, if needed

NCSX may be using CTH field-mapping hardware, so suggestions for improvement are in everyone's best interests.

Field-mapping set-up

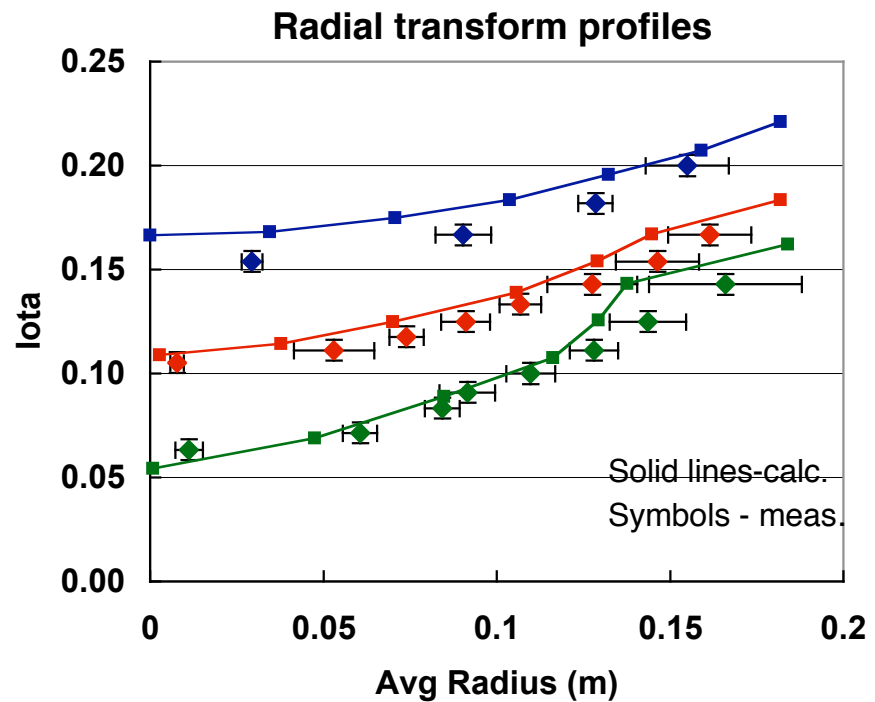
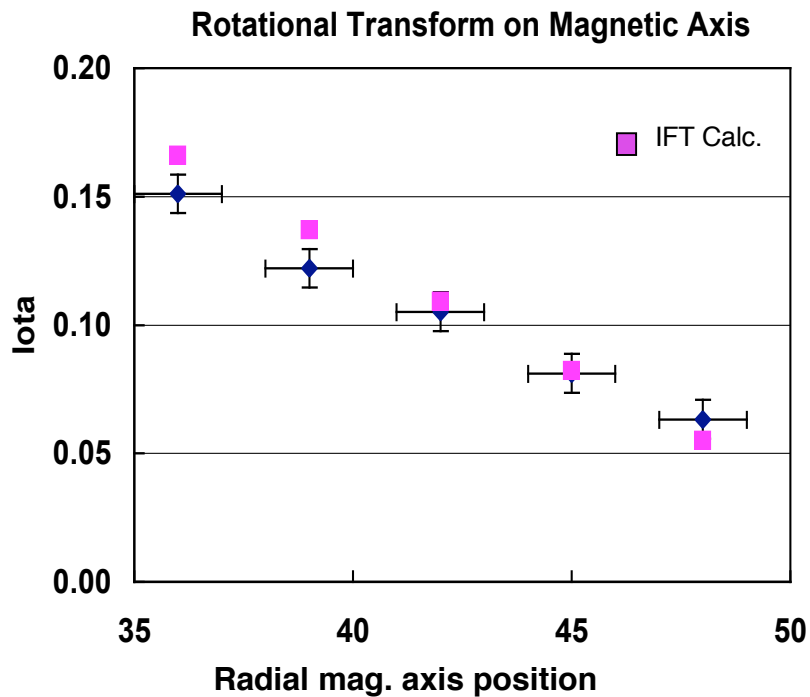


- Image photographed from either phosphor-coated mesh or movable wand
- Image processing software req'd for rotating effective field-of-view
- Most measurements made at one toroidal location; set-up recently rotated toroidally by two field periods

Surface-of-section for low rotational transform; no low-order rational surfaces
=> no islands

- Low aspect ratio configuration achieved

Field-mapping shows agreement with design



But some deviations from model observed ...

How can field mapping help us?

Test effect of inferred coil position errors

Modular & poloidal coil position & orientation errors;
dimensional tolerance on coils themselves

In our case, we have continuously-wound helical coil with winding law:

$$\phi = \underline{\frac{2}{5}\theta} + \alpha_1 \cos\theta + \alpha_2 \cos 2\theta + \underline{\beta_1 \sin\theta + \beta_2 \sin 2\theta} + \dots$$
$$\underline{r_{COIL}} = a_0 + a_1 \cos\theta + a_2 \cos 2\theta + b_1 \sin\theta + b_2 \sin 2\theta + \dots$$

ϕ : toroidal angle

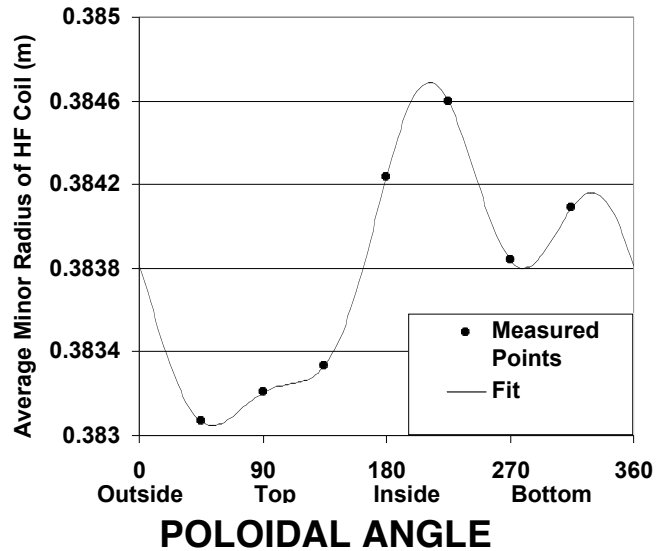
θ : toroidal angle

r_{coil} : helical coil minor radius = 0.385 m nominal

Underlined terms are non-zero in design

Deviations from helical coil winding law may explain observed discrepancies in field-mapping results

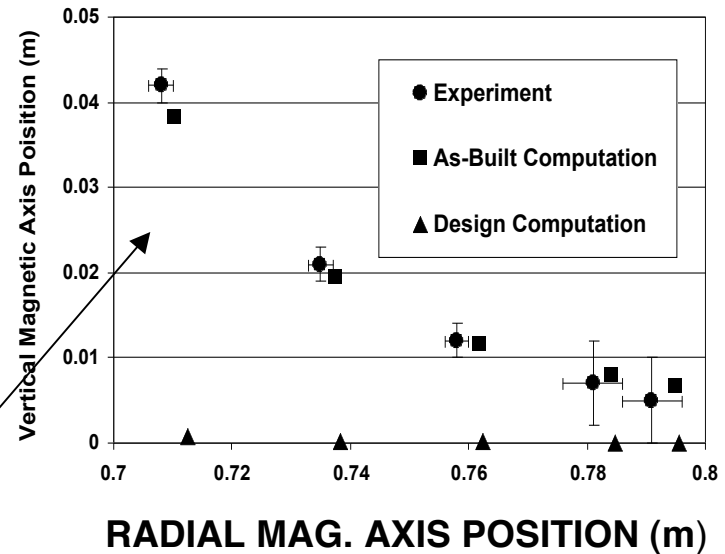
Average radius of helical coil measured during construction



⇒ Altered coil geometry to produce “as-built” coil model

Vertical asymmetry in magnetic axis position believed to be largely due to deviations of actual coil configuration from design

Scan of magnetic axis position in R-Z plane



As axis is shifted inward in this very low transform case, equilibrium bifurcates

Measured vertical shift equivalent in 2 different field periods; correctable with horizontal field

Can errors in coil placement be directly inferred from field-mapping?

Under investigation by Hanson & Munoz

- Use optimization procedure in Integrable Field Torsatron code to get derivatives of measurements with respect to parameters

Measurements: magnetic axis, fixed point locations in R, Z, rotational transform

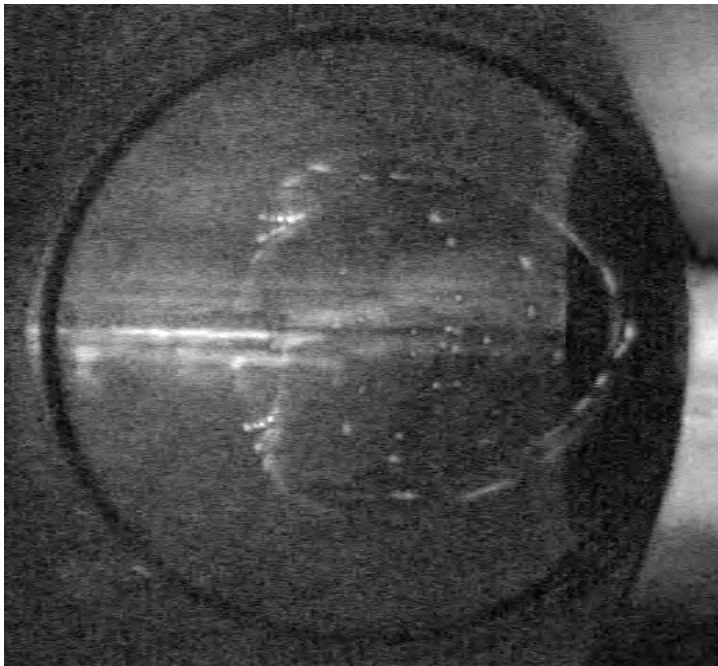
parameters: magnetic axis, rot. transform, coil current

- Obtain req'd change in parameters from SVD analysis

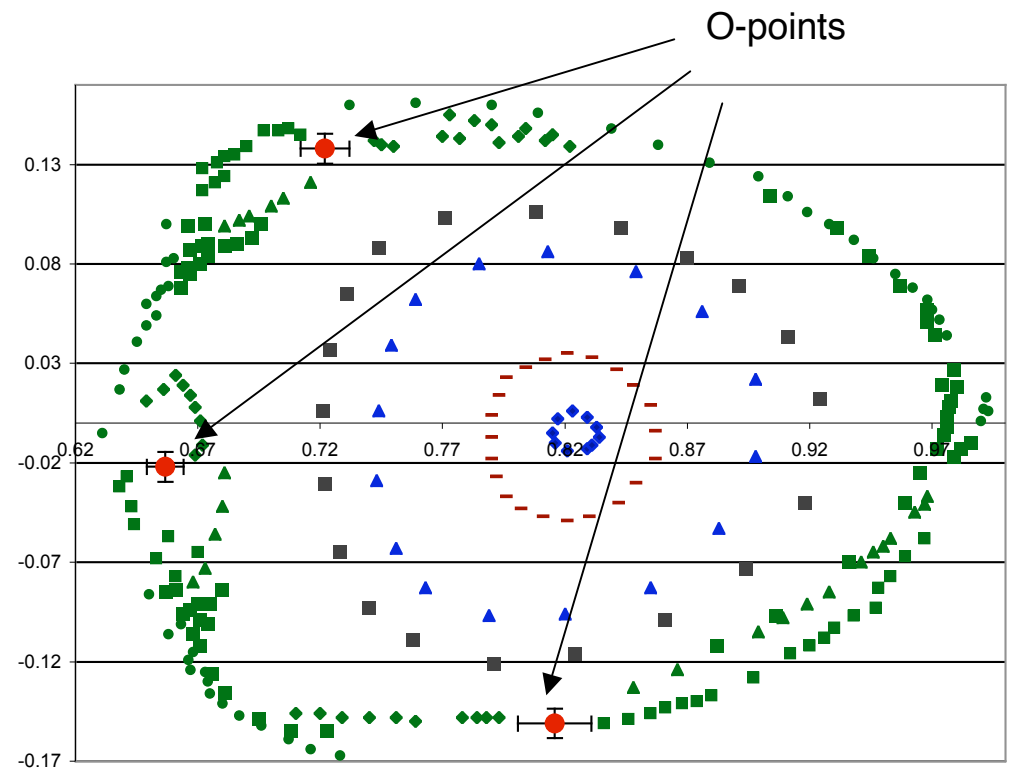
Magnetic islands observed on rational surfaces

With auxiliary TF coils, rotational transform raised to $\iota(a) \leq 0.7$

- $1/3$, $2/5$, & $1/2$ rational surfaces exhibit islands.



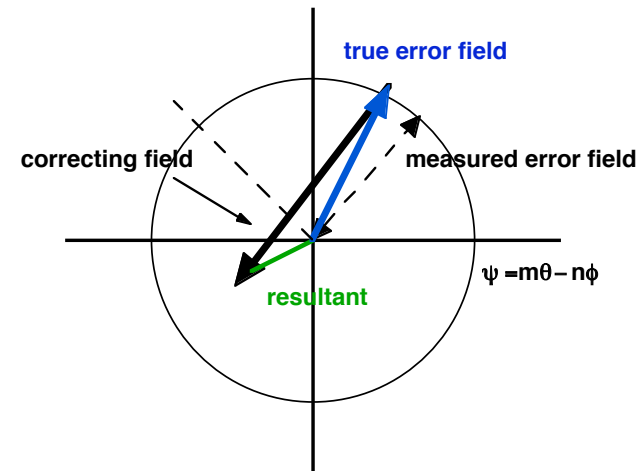
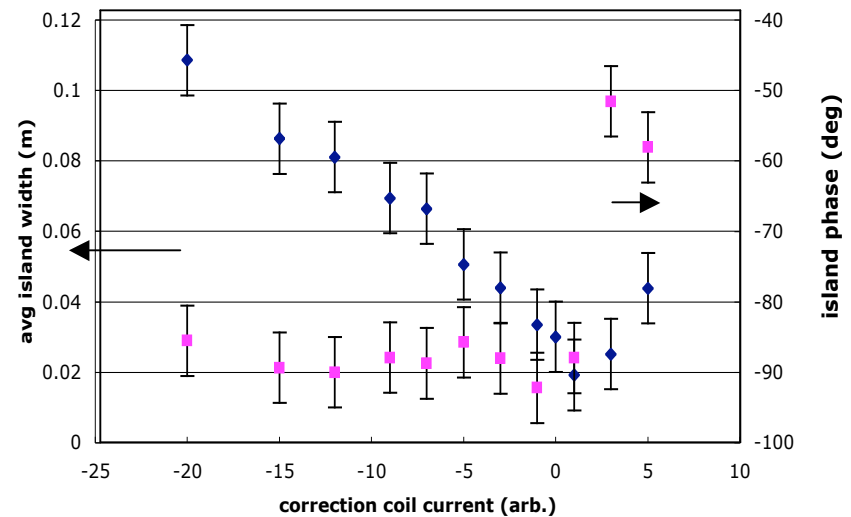
$n/m = 1/3$ island near the edge for $\iota(0) = 0.25$



Reducing static island sizes

Fixing static islands - m,n known

- Determine phase of island O or X-points $\psi = (m\theta_f - n\phi_f)$ in flux angle coords. (VMEC).
- Compute correction field of opposite phase; generate vector of N elements
N = no. of independent correction coils
- Apply correction vector of increasing magnitude.
Island size difficult to measure; phase v. sensitive to minimum size for given vector
- Continue minimization by applying add'l correction orthogonal to original vector at phase jump.
- Error correction procedure needs continued work
- Apply to correction of multiple islands (different rational surfaces in same equilibrium)
- What happens in stellarator plasma? (Reiman's stellarator theory presentation last May)



Near-term plans

- Complete mapping survey to include all coil sets w/ goal of improving vacuum field model as underpinning for 3-D reconstruction of plasmas

Better approaches for validation/improvement of coil model from vacuum field-mapping?

- Successfully validate procedure for vacuum island correction
- Plasma discharges diagnosed with in-vessel and ex-vessel magnetic diagnostics

Interface with existing and upcoming (V3FIT) reconstruction procedures

- Measure plasma flow, density, temperature in stochastic edge region
- Initial current-driven instability studies

Avenues for theory & modeling

- Modeling of kink and vertical instability thresholds for 3-D current-driven low- β CTH plasmas
- 3-D equilibrium reconstruction is a major challenge
- Island modification in presence of poloidal flow from ambipolar transport