List of anticipated research topics for the different phases of NCSX operation v0.2

NCSX Mission & Goals

Demonstrate:

Understand:

Conditions for high-beta, disruption-free operation.
Beta limits and limiting mechanisms. Reduction of neoclassical transport by QA design. Confinement scaling; reduction of anomalous transport by flow shear control. Equilibrium islands and neoclassical tearing- mode stabilization by choice of magnetic shear.
Compatibility between power and particle exhaust methods and good core performance.

Facility Upgrade Possibilities

	WBS	С
Heating; up to 3MW more NBI	25	
Heating; up to 6MW of RF	24	
Heating; pulse length up to 1-1.5 s.	24, 25	
Fueling: pellet injector	212	
Control: outboard trim coils	18, 4	
Control: 2T upgrade	4	
Control: system (e.g., more signals or computations)	55	
Control: magnet power supplies (faster ramps)	4	
P&PH: First wall #1 (e.g., add a slot, pump, and plenum baffles)	11	
P&PH: First wall #2 (e.g., optimized divertor)	11	Г
P&PH: Electrically biased first wall elements	11	
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'BS	Cost (r.o.m.)
25	
24	
, 25	
12	
3, 4	
4	
55	
4	
11	
11	
11	

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	P&PH: Wall conditioning upgrades (e.g. Lithiumization)	233]	
Phase	Торіс	Aux Power	Facility upgrades	Facility upgrade comments	Measurement Requirements
1. Initial Operation					
	initiate plasma control plasma evolution				plasma current conductivity plasma position plasma/wall imaging total stored energy line integrated density total radiated power central electron temperature impurity species Z<30
2. Field Line Mapping					
	map flux surfaces verify iota and QA				vacuum flux surfaces
3. Initial Ohmic					
	Improved plasma control, plasma evolution control		pellet injector	for density limit and profile studies.	electron temperature profiles
	global confinement & scaling, effect of 3D				electron density profiles
	snaping density limit & mechanisms with pellets? study of Te and ne profiles. vertical stability current-driven kink stability effect of low-order rational surfaces on flux- surface topology initial study of effect of trim coils, both signs effect of contact location on plasma edge & recycling initial attempts to control plasma contact				radiated power profiles low (m,n) MHD (<50kHz) flux surface topology Zeff hydrogen recycling
	effect of contact location on plasma edge & recycling initial attempts to control plasma contact location				

Phase	Торіс	Aux Power	Facility upgrades	Facility upgrade comments	Measurement Requirements
Progress toward NCSX goals	Test predictions about kink & vertical stablity (limiting mechanisms) Initial global scalaing data base. Needs more work.				
4. Initial Aux. Heating					
	Plasma control with NB heating and CD	3MW NBI	Control system	to cope with aux. Heating &optimize based on phase 3 results.	ion temperature profile
	confinement scaling w/ iota, B,		Outboard trim coils	to cope with wider range of profiles & optimize based on phase 3 results.	toroidal rotation profile
	local transport measurements test of quasi-symmetry on confinement and transport				poloidal rotation profile iota profile
	perturbative transport measurements density limits with heating density control with aux. Heating test of kink & balooning stability at moderate				fast ion loss ion energy distribution neutron flux IR imaging
	effect of shaping on MHD stability pressure effects on surface quality controlled study of neoclassical tearing using trim coils use of trim coils to minimize rotation damping				high frequency MHD(<5Mhz)
	initial study of Alfvenic modes w/ NB ions blip measurements of fast ion confinement and slowing down wall coatings with aux. Heating effect of contact location on plasma edge & recycling				

Phase	Торіс	Aux Power	Facility upgrades	Facility upgrade comments	Measurement Requirements
	initial attempts to control plasma contact location initial attempts to obtain enhance confinement regimes wall biasing effects on confinement				
5. Confinement & beta push					
	Stability tests at beta >~ 4%	3MW NBI + 6MW (NBI or RF)	P&PH #1 (divertor)	Locate slot based on Phase 4 results.	
	detailed study of beta limit scaling	,	6 MW of NB or RF	decide which based on Phase 4 results	
	detailed studies of beta limiting mechanisms		Control system	to cope with divertor & optimize based on phase 4 results.	
	disruption-free operating region at high beta Enh. Conf.: Hmode		Lithiumization Power supplies	faster ramp rates	
	Enhanced confinement, rotation effects		Biased first wall	confinement enhancement	
	Enh. Conf.: hot ion regimes Enh. Conf.: RI mode Enh. Conf.: pellets Scaling of local transport and confinement turbulence studies scaling of power or other thresholds for enhanced confinement ICRF wave propagation and damping (possible))	2T	low vu-star	
	ICRF heating of ions and electrons (possible)				

Phase	Торіс	Aux Power	Facility upgrades	Facility upgrade comments	Measurement Requirements
	perturbative RF measurements of transport (possible) active mapping of Alfvenic mode stability (with antenna) edge studies (2nd generation wall, e.g. divertor)				
	trace helium exhaust and confinement scaling of power to divertor evolution control of high beta plasmas				
6. Long Pulse					
	long pulse plasma evolution control	12 MW	3 MW of NB or RF	decide which based on Phase 5 results and depending on availability of long- pulse beams	
	equilibration of current profile	Long pulse	long-pulse		
	beta limits with ~ equilibrated profiles		P&PH #2	Re-contour based on Phase 5 results.	
	edge studies (3nd generation wall)		Control system	to cope with longer pulses and more	
	long-pulse power and particle exhaust handling				
	compatibility of high confinement, high beta, and divertor operation				